

Neutron Slowing Down Distances and Times in Hydrogenous Materials

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Outline

- Background / Lecture Material
 - Neutron Slowing Down Equation
 - Flux behavior in hydrogenous medium
- Fermi treatment for calculating slowing down distances and times.
- Brief derivation of mean square distance required to moderate neutron.
- Data from paper
- Average time required to moderate neutron
- Data from paper
- Monte Carlo Simulation (Explanation)
- Overview (What was learned?)
- References

Neutron Slowing Down

- The process of reducing the energy of a neutron from the fast to thermal energy range.
- To derive the neutron slowing down equation, first begin with the Neutron Transport Equation.

$$\frac{\partial n}{\partial t} = \int \sum_{dE d\Omega} (E) \phi(\bar{r}, E, \bar{\Omega}, t) F(E \bar{\Omega} \rightarrow E, \bar{\Omega}) + \frac{v f(E)}{4\pi} \int dE d\Omega \sum_{\downarrow} (E) \phi(\bar{r}, E, \bar{\Omega}, t) - \sum_{\uparrow} (E) \phi(\bar{r}, E, \bar{\Omega}, t) - \bar{v} \cdot \bar{\nabla} n(\bar{r}, E, \bar{\Omega}, t) + s(\bar{r}, E, \bar{\Omega}, t)$$

- By getting rid of the spatial dependence of the flux, we obtain the slowing down equation.

$$\sum_{\downarrow} (E) \phi(E) = S(E) + \int dE' \sum_{\downarrow} (E') \phi(E') F(E' \rightarrow E)$$

Neutron Slowing Down cont.

- The energy transfer kernel seen in the neutron slowing down equation is:

$$F(E' \rightarrow E) = \begin{cases} \frac{1}{E'(1-\alpha)} & \alpha E' \leq E \leq E' \\ 0 & \text{otherwise} \end{cases}$$

$$\alpha = \frac{(A-1)^2}{(A+1)^2}$$

Where the following assumptions are made:

- i.) elastic scattering
- ii.) target nucleus at rest
- iii.) isotropic scattering in CMCS

This kernel applies to neutron slowing down because the neutron loses energy each time it is scattered by the nucleus.

Flux behavior in Hydrogenous Mediums

- Begin with slowing down equation
- Let $A=1$, monoenergetic point source at E_o , neglect absorption, apply energy transfer kernel value.

$$S(E) = S_o \delta(E - E_o)$$

- $$\sum_s(E) \phi(E) = \frac{S_o}{E_o} + \int_E^{E_o} \sum_s(E') \phi(E') \frac{dE'}{E'}$$

$$G(E) = \text{const} + \int_E^{E_o} G(E') \frac{dE'}{E'}$$

$$G(E) = \frac{c}{E} = \frac{S_o}{E}; c = S_o$$

$$\phi(E) = \frac{S_o}{\sum_s(E) E}$$

Fundamental Question and Issues

- How far from a point source does a neutron of initial energy E random walk before its energy is moderated to some lower energy E' ? ($E > E'$)
- Fermi addressed this problem by restricting the neutron mean free path, λ , to a constant or slowly varying with energy.
- However, the neutron scattering c.x. of ^1H is neither constant nor slowly varying in the energy range $0.1 \text{ MeV} < E < 14 \text{ MeV}$.

Accounting for Energy Dependence

- To determine how far a neutron wanders before it reaches a given lower energy, the Fermi treatment is manipulated such that λ is now energy dependent.
- This method is valid as long as the dominant neutron scattering mechanism is elastic and the mean free path is known.

Net Displacement

- The neutron will traverse a sequence of linear displacements \mathbf{r}_j , each terminated by scattering event that sends the neutron off in a new direction. After n^{th} path the net displacement, \mathbf{R} , of the neutron from the origin is given by:

$$\mathbf{R} = \sum_{j=1}^n \mathbf{r}_j$$

Image removed due to copyright considerations.

Please see:

Net Displacement figure taken from: Nellis, W. J. "Slowing-down distances and times of 0.1- to 14- MeV neutron in hydrogenous materials." *American Journal of Physics* 45, no. 5 (May 1977).

Brief Derivation

- Magnitude Squared of the Net Displacement, \mathbf{R} , is R^2 .

$$R^2 = \sum_{j=1}^n r_j \cdot \sum_{k=1}^n r_k$$

$$\langle R^2 \rangle = \sum_{j=1}^n \langle r_j^2 \rangle + 2 \sum_{j=1}^{n-1} \sum_{k=j+1}^n \langle r_j \rangle \langle r_k \rangle \langle \cos \theta_{jk} \rangle$$

Where θ_{jk} is the angle between the j^{th} and the k^{th} paths. Neutrons scatter with radial distribution function $e^{-r/\lambda}$, where λ is the total mean free path.

$$\langle r_j \rangle = \lambda_j \quad \text{and} \quad \langle r_j^2 \rangle = 2\lambda_j^2$$

$$\langle \cos \theta_{jk} \rangle = \langle \cos \theta \rangle^{k-j} = \mu^{k-j}$$

Derivation cont.

- Thus, if λ is constant, then

$$\langle R^2 \rangle = \frac{2n\lambda^2}{1-\mu} \left[1 - \frac{\mu}{n} \left(\frac{1-\mu^n}{1-\mu} \right) \right]$$

- But if λ varies with energy, then

$$\langle R^2 \rangle = 2 \left[\sum_{j=1}^n \lambda_j^2 + \sum_{j=1}^{n-1} \lambda_j \sum_{k=j+1}^n \mu^{k-j} \right]$$

The root mean squared slowing down distance is:

$$\langle R_{rms} \rangle = \langle R^2 \rangle^{1/2}$$

Data: 14 MeV Neutron Moderated in Hydrogen ($N = 8 \times 10^{22} \text{cm}^{-3}$)

Image removed due to copyright considerations.

Please see:

Neutron Moderated in Hydrogen table taken from: Nellis, W. J. "Slowing-down distances and times of 0.1- to 14- MeV neutron in hydrogenous materials." *American Journal of Physics* 45, no. 5 (May 1977).

Slowing Down Time

- Average time, t_{ave} , required to moderate neutron from high to lower energy is:

$$t_{ave} = \sum_{j=1}^n \Delta t_j = \sum_{j=1}^n \frac{\lambda_j}{v_j}$$

Where Δt is the time spent at each energy.

In the figure, 14 MeV neutrons are being moderated to Ef.

Image removed due to copyright considerations.

Please see:

Slowing Down Time graph taken from: Nellis, W. J. "Slowing-down distances and times of 0.1- to 14- MeV neutron in hydrogenous materials."

American Journal of Physics 45, no. 5 (May 1977).

Monte Carlo Simulation

- The sample problem consists of:
 - 1 MeV point source
 - Neutrons emitted with delta function time distribution
 - Point source at the center of the hydrogen sphere
 - Hydrogen density $N = 8 \times 10^{22} \text{ cm}^{-3}$
 - Hydrogen sphere has an outer radius of 50 cm
 - Path lengths are chosen randomly from a distribution weighted by total c.x.
 - Scattering angle are chosen randomly from a distribution weighted by the known angular dependence of the c.x.
 - Local energy deposited in a collision is then calculated based on conservation of energy and momentum.
 - Flux tallies were placed at different radiuses in hydrogen sphere so one can monitor the moderation.
 - E_f was set to be 40 ev

Overview

- As energy increases, the hydrogen c.x. decreases; thus, the total mean free path is larger. --- Higher energy neutrons penetrate deeper than lower energy neutrons.

$$\lambda_T = \frac{1}{N \sigma_T}$$

- The higher the neutron energy, the faster the neutron loses energy.
- 1/E flux behavior is characteristic of the neutron distribution during slowing down by elastic collisions with target.
- * Hydrogen is the most efficient neutron moderator since it can absorb the largest fraction of neutron energy per elastic collision.

References

- Nellis, W.J., ***Slowing-down distances and times of 0.1- to 14- MeV neutron in hydrogenous materials.*** American Journal of Physics, Vol.45, No.5, May 1977.
- J.R. Lamarsh, ***Introduction to Nuclear Reactor Theory.*** © 1966
- Neutron Interactions and Applications, Lecture 9: Neutron Slowing Down.