## Problem Set 7 <br> Due SES \#20

[EL] = Lewis, Elmer L. Fundamentals of Nuclear Reactor Physics. Burlington, MA:
Academic Press, 2008. ISBN: 9780123706317.

Suggested Problems: [EL] Chapter 7, Problems 7.5, 7.13

Question 1: Determine the critical radius of a sphere using:
a) 1-group theory
b) 2-group theory

Table 1.1: 1-group parameters

| $\mathrm{D}(\mathrm{cm})$ | 1 |
| :---: | :---: |
| $\Sigma_{\mathrm{a}}\left(\mathrm{cm}^{-1}\right)$ | 0.25 |
| $\mathrm{v} \Sigma_{\mathrm{f}}\left(\mathrm{cm}^{-1}\right)$ | 0.3125 |

Table 1.2: 2-group parameters

| $\mathrm{D}_{1}(\mathrm{~cm})$ | 2 |
| :---: | :---: |
| $\Sigma_{\mathrm{r} 1}\left(\mathrm{~cm}^{-1}\right)$ | 0.08 |
| $\nu \Sigma_{\mathrm{f} 1}\left(\mathrm{~cm}^{-1}\right)$ | 0.02 |
| $\mathrm{D}_{2}\left(\mathrm{~cm}^{-1}\right)$ | 1 |
| $\Sigma_{\mathrm{a} 2}\left(\mathrm{~cm}^{-1}\right)$ | 0.25 |
| $\Sigma_{\mathrm{f} 2}\left(\mathrm{~cm}^{-1}\right)$ | 0.5 |
| $\Sigma_{\mathrm{s} 21}$ | 0.058 |

Question 2: Write the 4 group steady-state diffusion equations with no external source. Assume that all fission neutrons are born in the fast group and that there is noupscattering.

If we assume that the leakage in groups 2,3 and 4 is negligible compared to the collision terms in these groups, derive the corresponding modified 1 group equation.

Question 3: A bare spherical reactor is to be constructed of a homogeneous mixture of $\mathrm{D}_{2} \mathrm{O}$ and U-235. The composition is such that for every uranium atom there are 2000 heavy water molecules (i.e. $\mathrm{N}_{\mathrm{D} 2 \mathrm{O}} / \mathrm{N}_{\mathrm{U}-235}=2000$ ). Calculate the critical radius of the reactor using one-speed diffusion theory using the following parameters:

$$
\begin{aligned}
& \eta_{\mathrm{U}-235}=2.06 \\
& \mathrm{D}_{\mathrm{D} 2 \mathrm{O}}=0.87 \mathrm{~cm} \\
& \Sigma_{\mathrm{aD} 2 \mathrm{O}}=3.3 \times 10^{-5} \mathrm{~cm}-1 \\
& \sigma_{\mathrm{a}, \mathrm{D} 2 \mathrm{O}}=0.001 \text { barn } \\
& \sigma_{\mathrm{a}, \mathrm{U}-235}=678 \text { barn }
\end{aligned}
$$

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### 22.05 Neutron Science and Reactor Physics

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